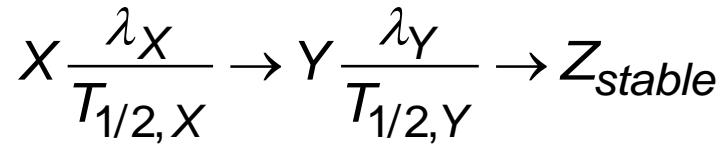
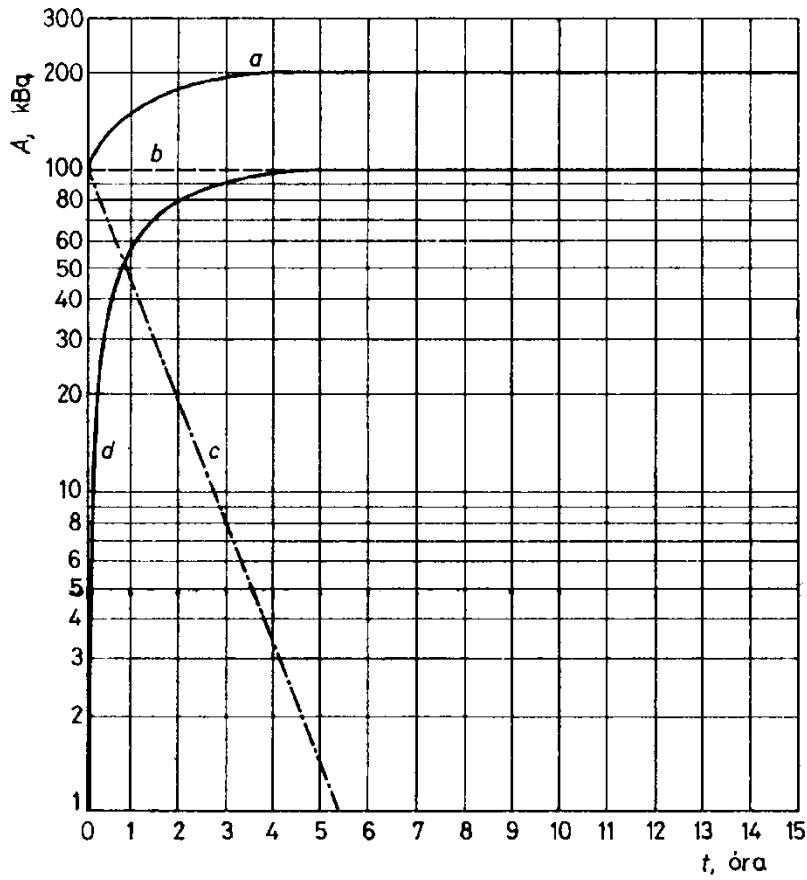


## Decay chains



$$A_Y = \lambda_Y N_Y = A_{X,0} \frac{\lambda_Y}{\lambda_Y - \lambda_X} \left( e^{-\lambda_X t} - e^{-\lambda_Y t} \right)$$

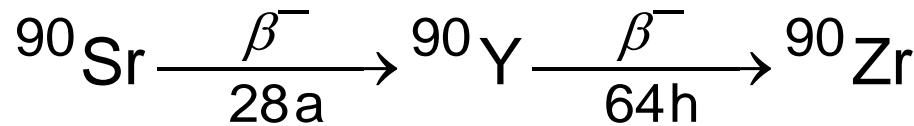
relation of  $\lambda_A$  and  $\lambda_B$  ?

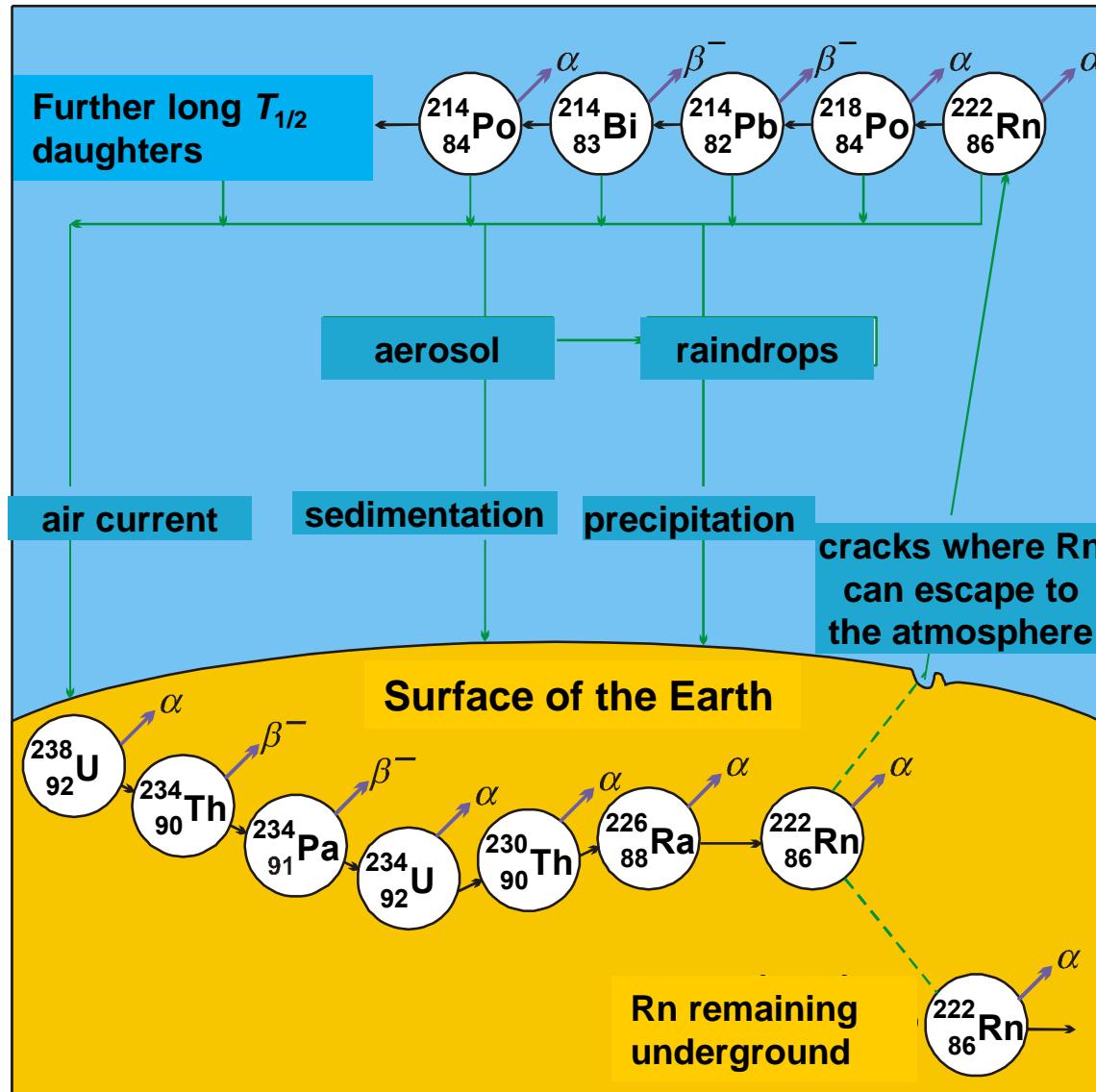
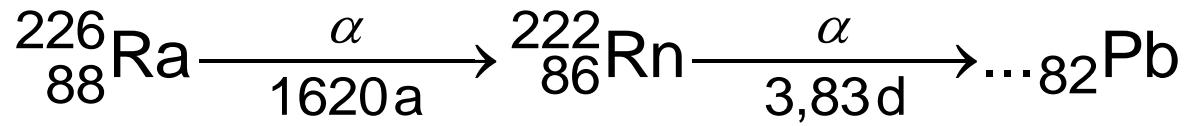


$$T_{1/2,X} \gg T_{1/2,Y}$$

$$T_{1/2,X} = 8 \cdot 10^7 \text{ h}$$

$$T_{1/2,Y} = 0,8 \text{ h}$$





When former Russian spy Alexander Litvinenko died from polonium-210 poisoning several years ago in London, it triggered a murder investigation that developed like a thriller.

Po-210 generate much heat as the atoms decay - it was used in Russian lunar landers to keep the craft's instruments warm at night.

$^{210}\text{Po}$  is an  $\alpha$ -emitter, that has a half-life of 138.4 days,  $E_\alpha = 5.3 \text{ MeV}$

# Interaction of the radiation with the matter

## Particles/photons

I.	b	II.	III.
a			
p	$e^+$	n	$\gamma$

$\alpha$

$e^-$

X

## Partners

1. Electromagnetic field
2. Electron
3. Field of the nucleus
4. Nucleus

## Mechanism

A) Absorption

Effect on  
radiation      matter  
 $\Delta I$                $E_{kin}, E^*$

B) Coherent scattering (only the direction  
is altered))

$\Delta I$               -

C) Incoherent scattering (also exchange of  $E$ )  
elastic (no excitation)  
inelastic

$\Delta I, \Delta E$   
 $E_{kin}$   
 $E_{kin}, E^*$       6

# 1. Ionizing radiations

# The first step of the ionizing radiation in the matter:

1. Neutral excitation



2. External ionization



3. Internal ionization

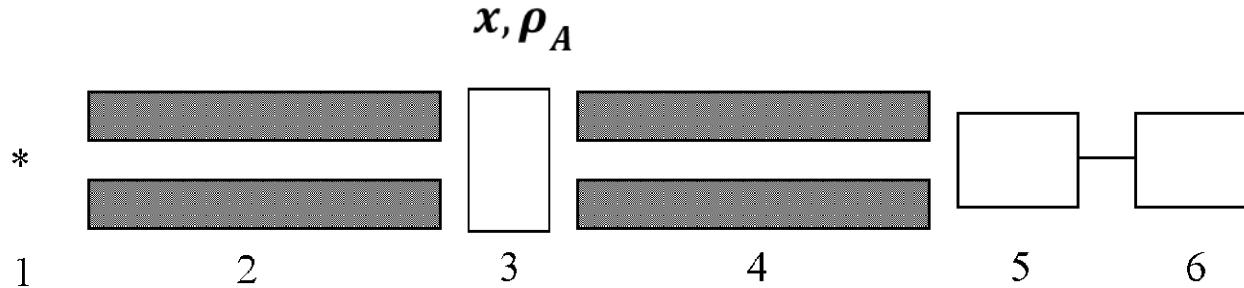


4. Bremsstrahlung (breaking radiation)



## FUNDAMENTALS OF DETECTION

# Quantitative description of the interaction



$$v = \sigma n x \rho_A$$

cross section

$$-dn = \sigma(E) n \rho_A dx$$

$$n = n_0 e^{-\sigma(E) \rho_A x}$$

$$I = \frac{n}{t}$$

$$I = I_0 e^{-\mu x}$$

linear absorption coefficient

$$I = I_0 e^{-\mu x} = I_0 e^{-\frac{\rho}{\rho} x \cdot \rho} = I_0 e^{-\mu_m d}$$

mass absorption  
coefficient

$$x_{1/2} = \frac{\ln 2}{\mu} \quad d_{1/2} = \frac{\ln 2}{\mu_m}$$

## $\alpha$ -radiation

Heavy, charged, high energy

With electrons: incoherent scattering

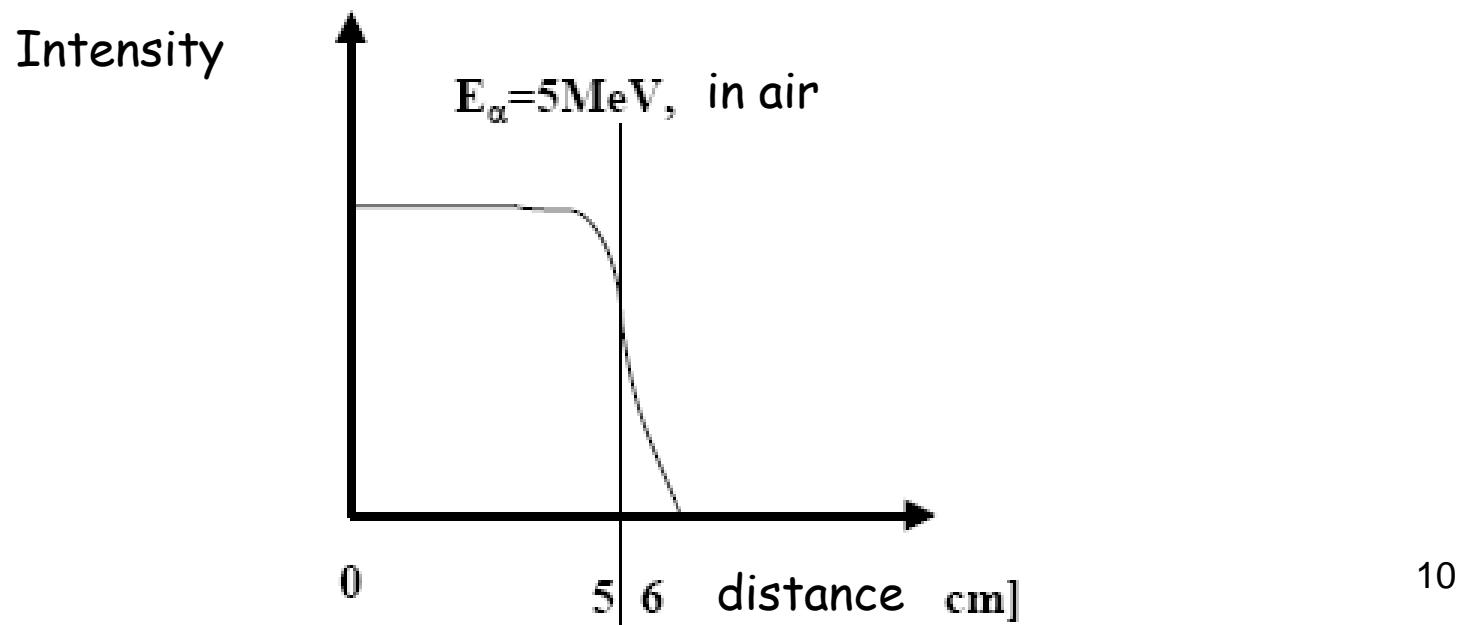
ionisation and excitation (50-50 %)

$E$  and direction of the alpha particles is modified

With the nucleus: Rutherford-scattering

nuclear reaction (see later)

! Bremsstrahlung (continuous energy gamma radiation)!



$\beta$ -radiation small, charged, limited energy

With electron: incoherent scattering

ionisation (external and internal)

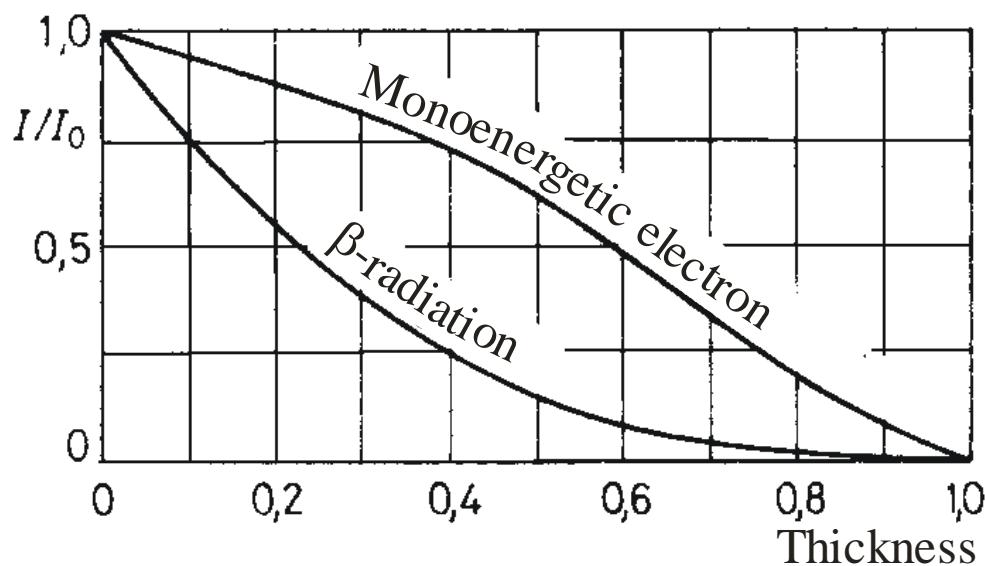
excitation

$E$  and the direction of the radiation changes

$$\frac{\left(\frac{dE}{dx}\right)_r}{\left(\frac{dE}{dx}\right)_{\text{ion}}} = \frac{EZ}{800}$$

With the field of the nucleus: incoherent scattering

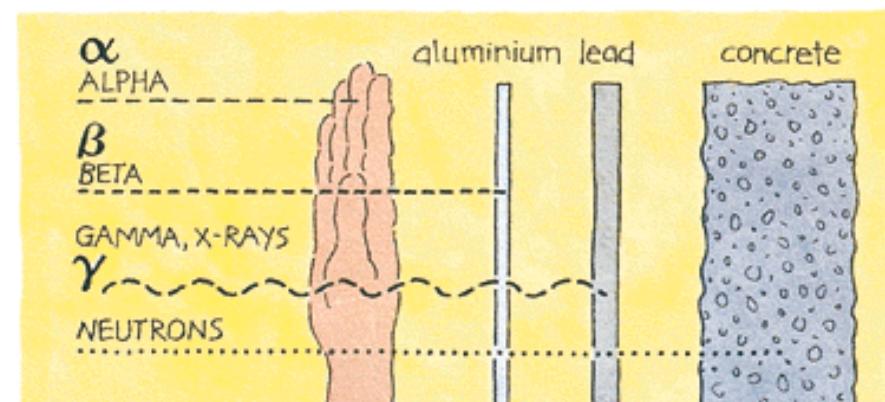
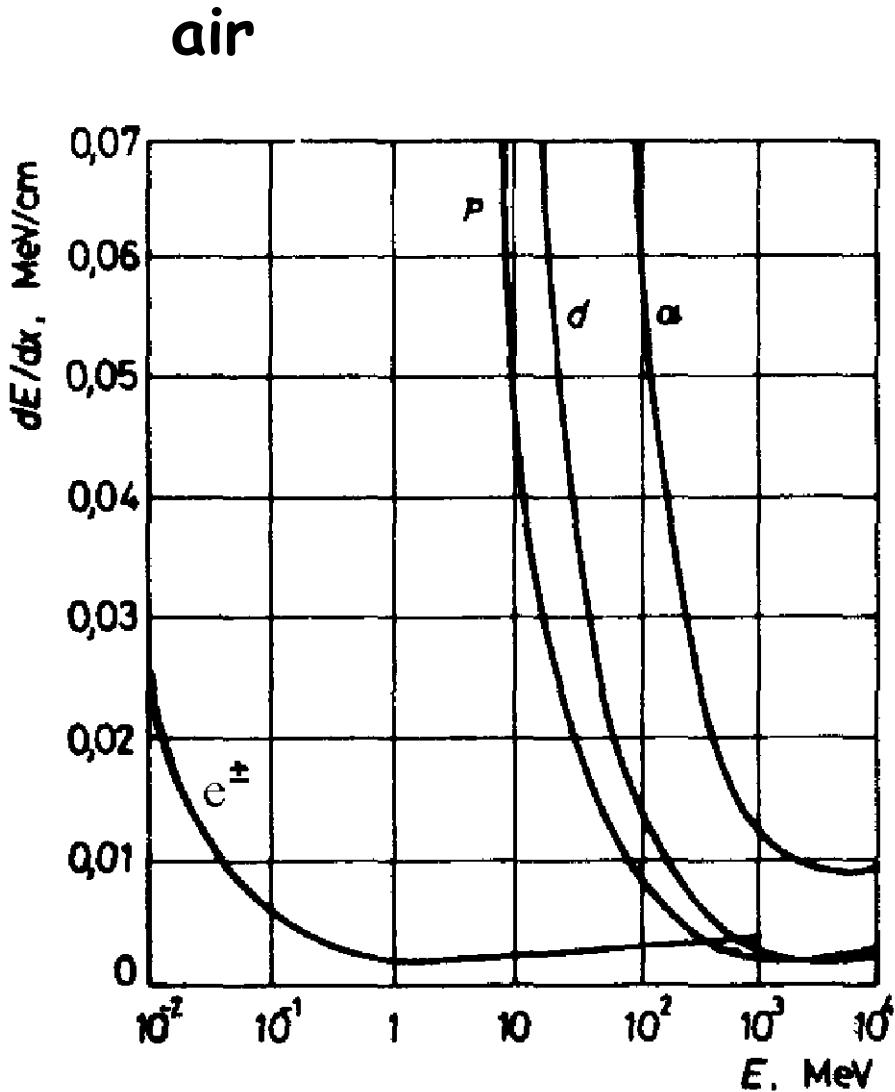
! Bremsstrahlung !



$$I = I_0 e^{-\mu x} = I_0 e^{-\mu_m d}$$

Linear/mass absorption coefficient <sup>11</sup>

# Linear energy transfer (LET)



$$dE/dx \approx 1/v^2$$

Calculate the activity of 1 kg KCl. 0.012 % of the K atoms is radioactive  $^{40}\text{K}$ . The half life of  $^{40}\text{K}$  is  $1.13 \cdot 10^9$  years.

We prepared a  $^{35}\text{S}$  labelled protein at 12:00, 10 September 2014. The half life of the pure  $\beta^-$  emitter is 88 days. This sample was measured at noon on 26 September and the intensity was found 7000 imp/s. The overall efficiency of the measurement was 22 %. Calculate the activity of the sample in the time of synthesis.

The linear absorption coefficient of gamma radiation of 660 keV in aluminum is  $3,4 \text{ cm}^{-1}$ . Calculate the half thickness. How efficiently will attenuate this radiation an 10 cm aluminum wall ?

## $\gamma$ -radiation

## electromagnetic radiation

### 1. Compton-scattering

Elastic collision of the photon with an electron

$E_C$

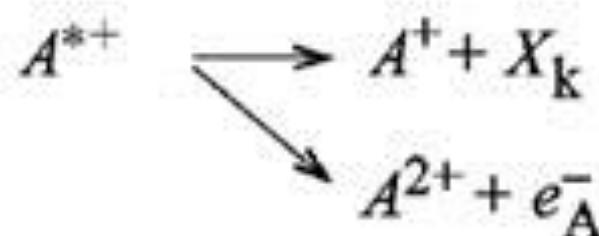
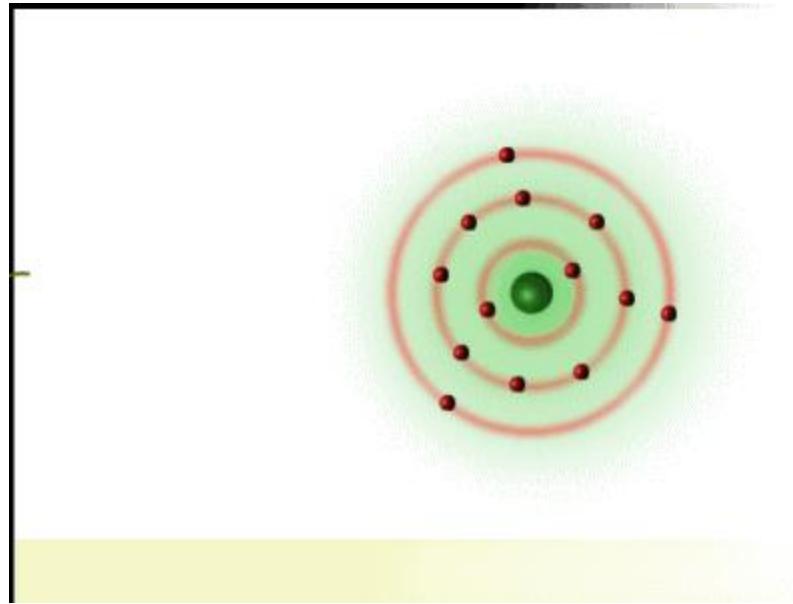


$E'_\gamma$

$$\mu_{C,m} = \frac{\mu_C}{\rho} = \sigma_C \frac{\rho_A}{\rho} = \sigma_C \frac{N_A Z}{A}$$

where  $\sigma_C = \sigma_s + \sigma_a$

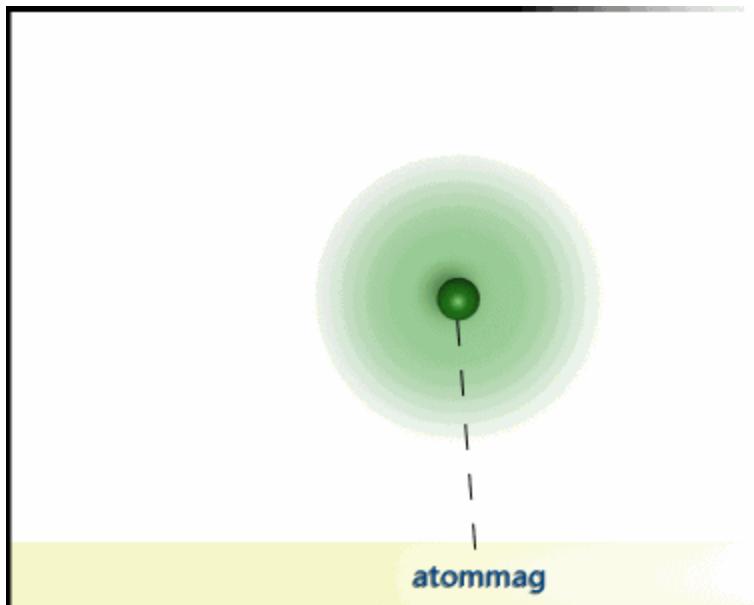
## 2. Photoelectric effect



$$\sigma_f \approx \text{konst.} \frac{Z^n}{(h\nu)^3}$$

$$n(E)=4 - 5$$

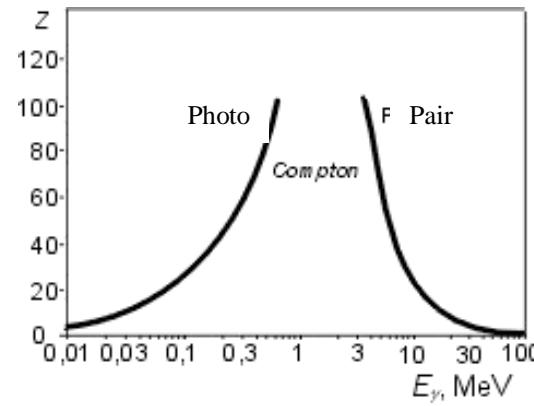
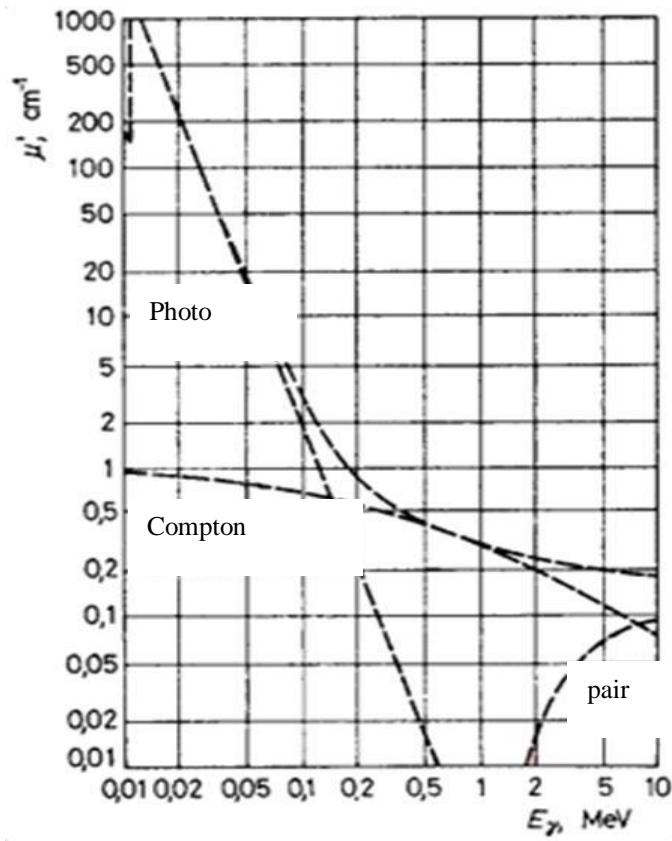
### 3. Pair production



$$\sigma_p = K(E_p - 1,02)^{2,2} Z^2$$

$$I = I_0 e^{-\mu d} = I_0 e^{-(\mu_C + \mu_f + \mu_p)d}$$

## Germanium



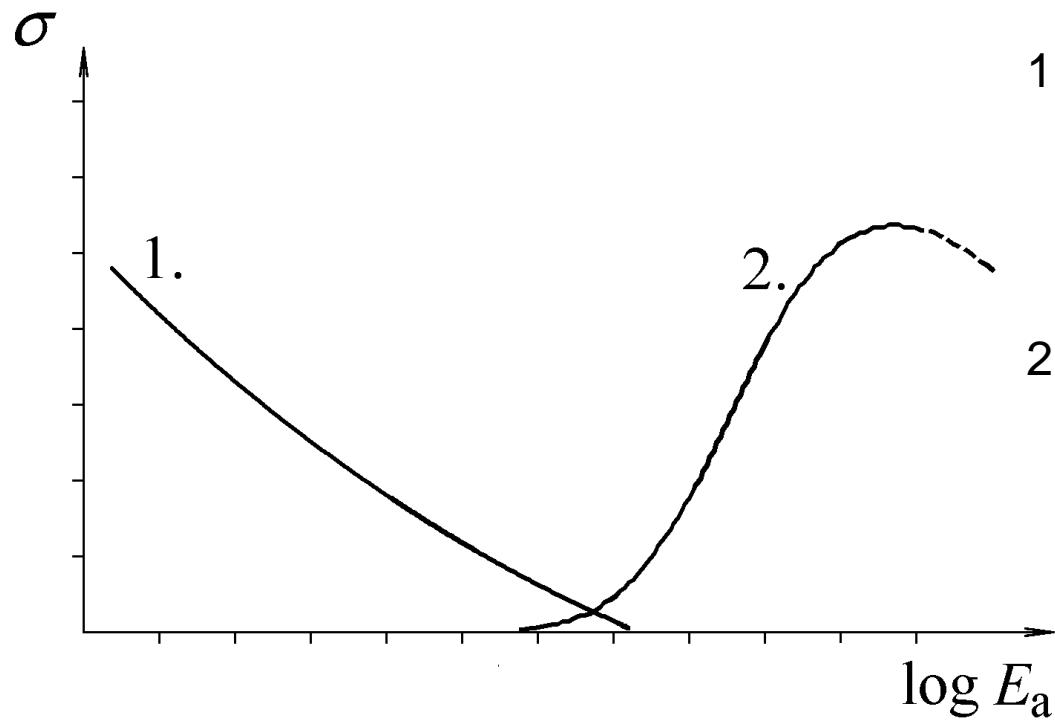
## 2. Nuclear reactions

# Cross section (~probability)

Conventional equation



Transition state



1.

(n,γ)  
(n,f)  $^{233}\text{U}$ ,  $^{235}\text{U}$ ,  $^{239}\text{Pu}$ ,  $^{241}\text{Pu}$   
 $^{10}\text{B}(n,\alpha)$   
 $^{6}\text{Li}(n,\alpha)$

2.

(γ,n)  
(n,2n)  
(n,α)  
(p, )  
(d, )

Tunnel effect

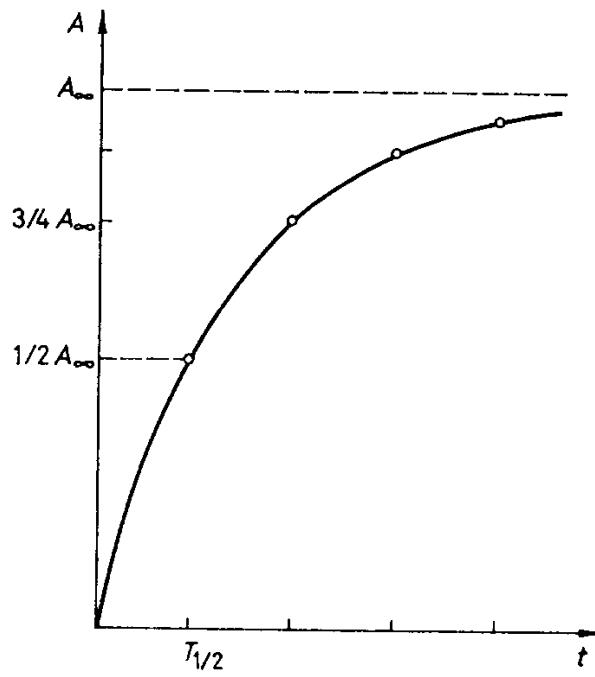
# Kinetics of the nuclear reactions

$$\frac{dN^*}{dt} = \sigma_a N \phi - \lambda N^*$$

$$N^* = N_\infty^* [1 - \exp(-\lambda t)]$$

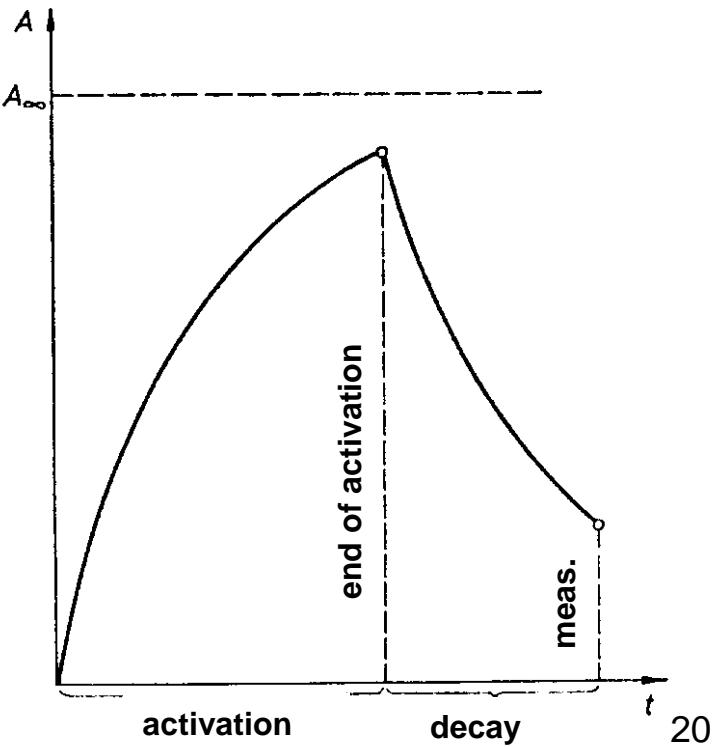
$$A = A_\infty^* [1 - \exp(-\lambda t)]$$

$$A_\infty^* = \lambda N_\infty^* = \phi \sigma_a N$$



$$A' = \lambda N^* =$$

$$= A_\infty^* [1 - \exp(-\lambda t)] \exp(-\lambda t_h)$$



We intend to obtain  $^{65}\text{Ni}$  with neutron irradiation. Therefore, we expose 1 g of Ni (with a  $^{64}\text{Ni}$  content of 91 %) to neutrons with a flux  $\Phi = 10^{12} \text{ 1/cm}^2\text{s}$ . The cross section  $\sigma$  of the



reaction is  $1.55 \cdot 10^{-28} \text{ m}^2$ . The half-life of  $^{65}\text{Ni}$  is 2.52 h.

- i) How long should the irradiation last if we want to reach 80 % of the saturation activity?
- ii) Estimate the ratio of the  $^{64}\text{Ni}/^{65}\text{Ni}$  isotopes in the sample after being „cooled“ for the same period as the activation lasted.

# Interaction of neutrons with the matter

## relatively heavy, no charge, energy ?

- elastic scattering

Table R8. The energy absorption efficiency of light elements

( $E_0 = 2 \text{ MeV}$ ,  $E = kT$ )

Element	$\Delta\bar{E}$ , keV	$n$
$^1\text{H}$	1000	18
$^2\text{D}$	888	24
$^4\text{He}$	640	41
Be	360	50
C	284	111
Al	137	240

- inelastic scattering

Excited nucleus,  $h\nu$

- neutron capture

(absorption): ( $n, ?$ )

## Due to the strong $E$ dependence,

1. Slow			
a) cold		$E <$	0.025 eV
b) <i>thermal</i>	0.025 eV	$< E <$	0.44 eV
c) resonance	0.44 eV	$< E <$	1000 eV
2. Medium	1 keV	$< E <$	500 keV
3. Fast	0.5 MeV	$< E <$	10 MeV
4. High energy	10 MeV	$< E <$	50 MeV
5. Super fast	50 MeV	$< E$	

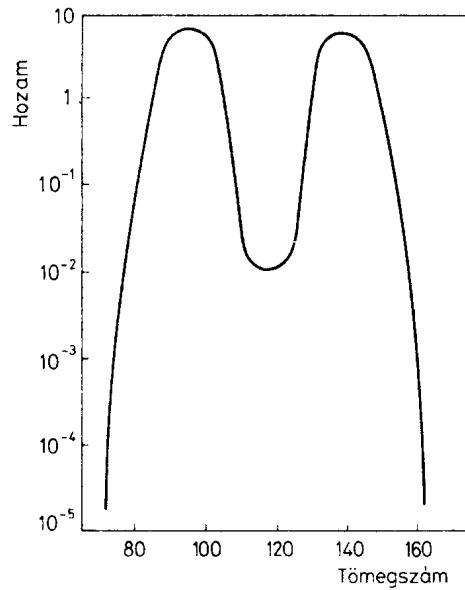
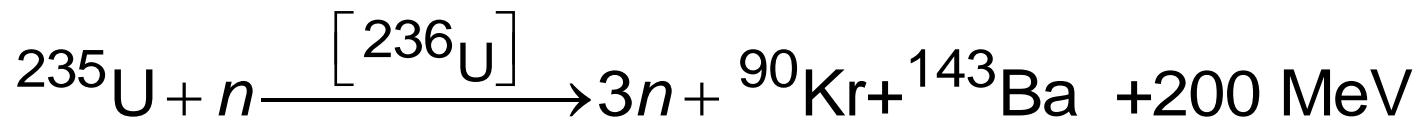
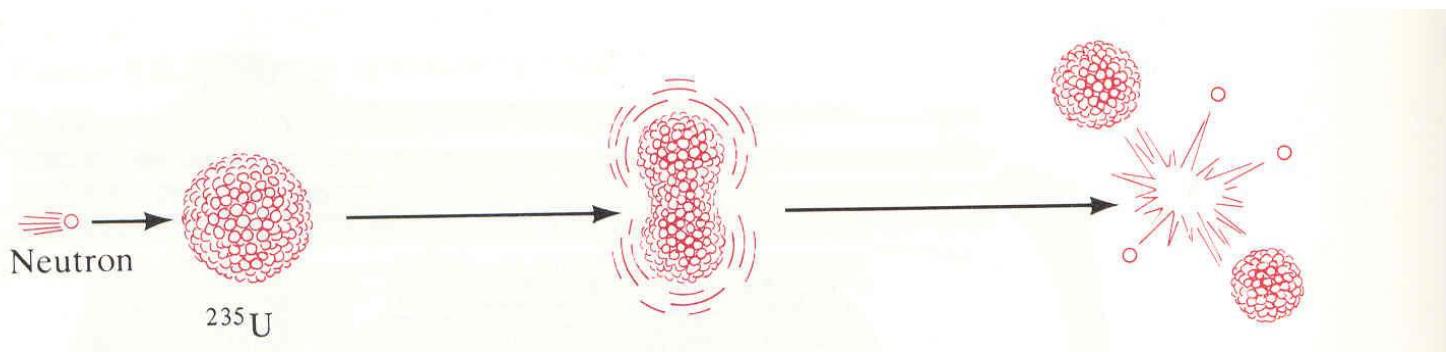
## Examples of practical relevance

$(n, \gamma)$	$^{113}\text{Cd}(n, \gamma)^{114}\text{Cd}$	$\sigma = 6,31 \cdot 10^{-24} \text{ m}^2$
	$^{135}\text{Xe}(n, \gamma)^{136}\text{Xe}$	$\sigma = 2,7 \cdot 10^{-22} \text{ m}^2,$
	$^{149}\text{Sm}(n, \gamma)^{150}\text{Sm}$	$\sigma = 6,6 \cdot 10^{-24} \text{ m}^2,$
	$^{157}\text{Gd}(n, \gamma)^{158}\text{Gd}$	$\sigma = 4,6 \cdot 10^{-23} \text{ m}^2,$
$(n, \alpha)$	$^{10}\text{B}(n, \alpha)^7\text{Li}$	$\sigma = 3 \cdot 10^{-25} \text{ m}^2$

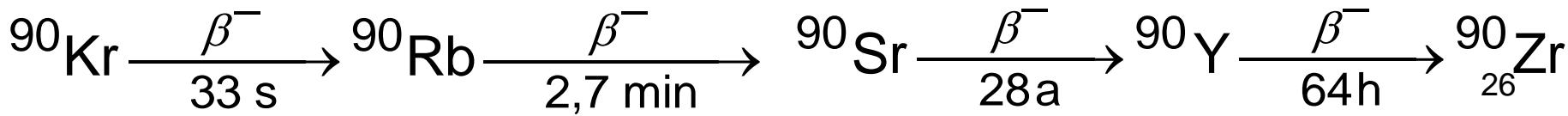
# $(n,f)$ fission

Fuel	Source of the fuel	Neutron energy needed
$^{235}\text{U}$	natural uranium	$0.025 \text{ eV} < E_{\text{neutron}} < 0.44 \text{ eV}$ (thermal)
$^{233}\text{U}$	from thorium with n absorption	$0.025 \text{ eV} < E_{\text{neutron}} < 0.44 \text{ eV}$ (thermal)
$^{239}\text{Pu}$	from $^{238}\text{U}$ with n absorption	$0.025 \text{ eV} < E_{\text{neutron}} < 0.44 \text{ eV}$ (thermal)
$^{241}\text{Pu}$	from $^{238}\text{U}$ with n absorption	$0.025 \text{ eV} < E_{\text{neutron}} < 0.44 \text{ eV}$ (thermal)
$^{238}\text{U}$	natural uranium	$0.5 \text{ MeV} < E_{\text{neutron}} < 10 \text{ MeV}$ (fast)
$^{232}\text{Pu}$	natural thorium	$0.5 \text{ MeV} < E_{\text{neutron}} < 10 \text{ MeV}$ (fast)

# Fission ( $n, f$ )



50 ways, 300 isotopes 35 elements



# Distribution 200 MeV

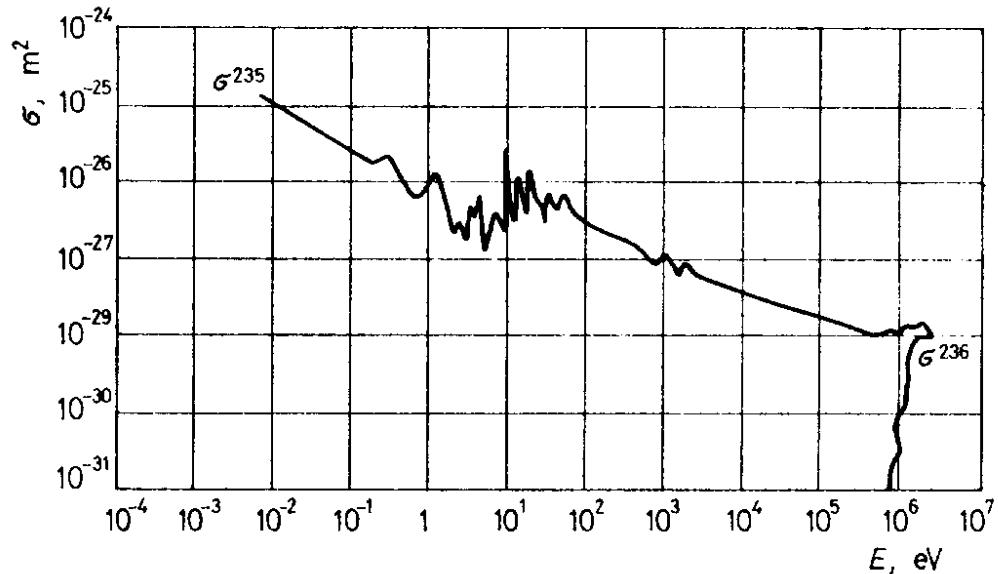
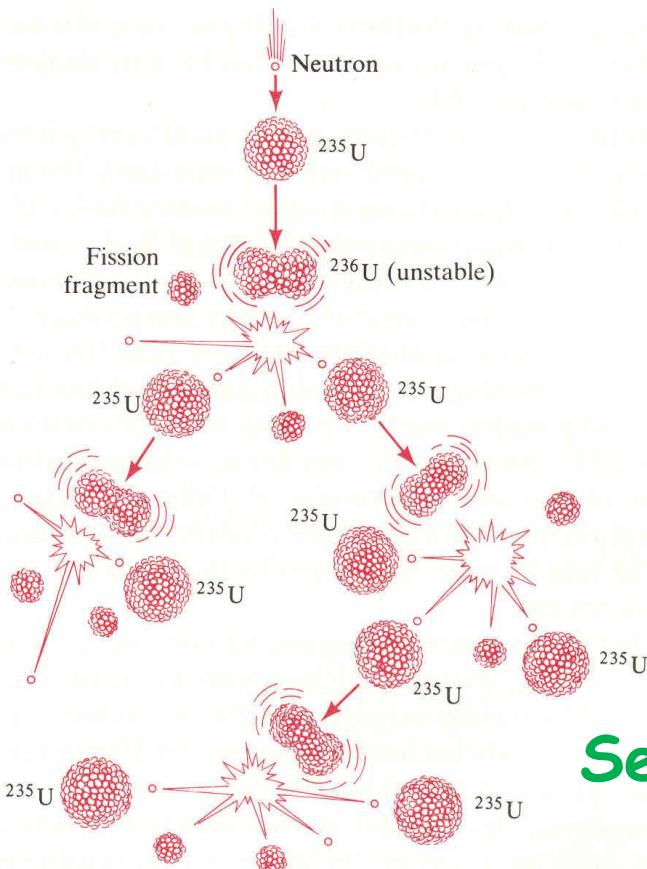
kinetic energy of fission products:  $\approx 160$  MeV

kinetic energy of the neutrons:  $\approx 5$  MeV

energy of the  $\gamma$ -rays:  $\approx 5$  MeV

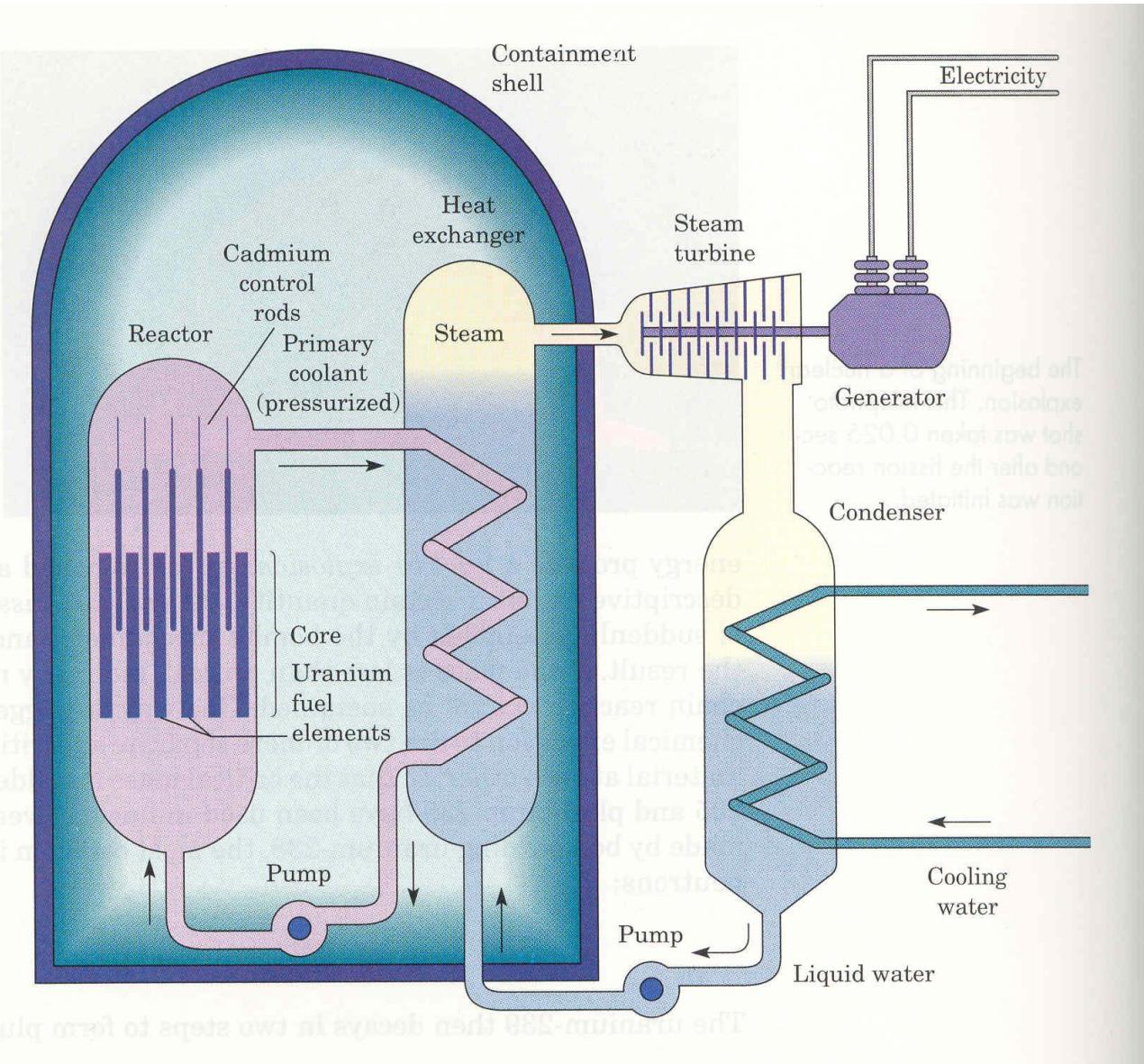
energy of the secondary radioactive decay:  $\approx 20$  MeV

energy released at neutron capture:  $\approx 10$  MeV



**Self-sustaining chain reaction: control**

# Nuclear reactor



Fuel  
Moderator  
Cooling system  
Control